

# Feasibility of determining spent fuel characteristics from neutron activation analysis

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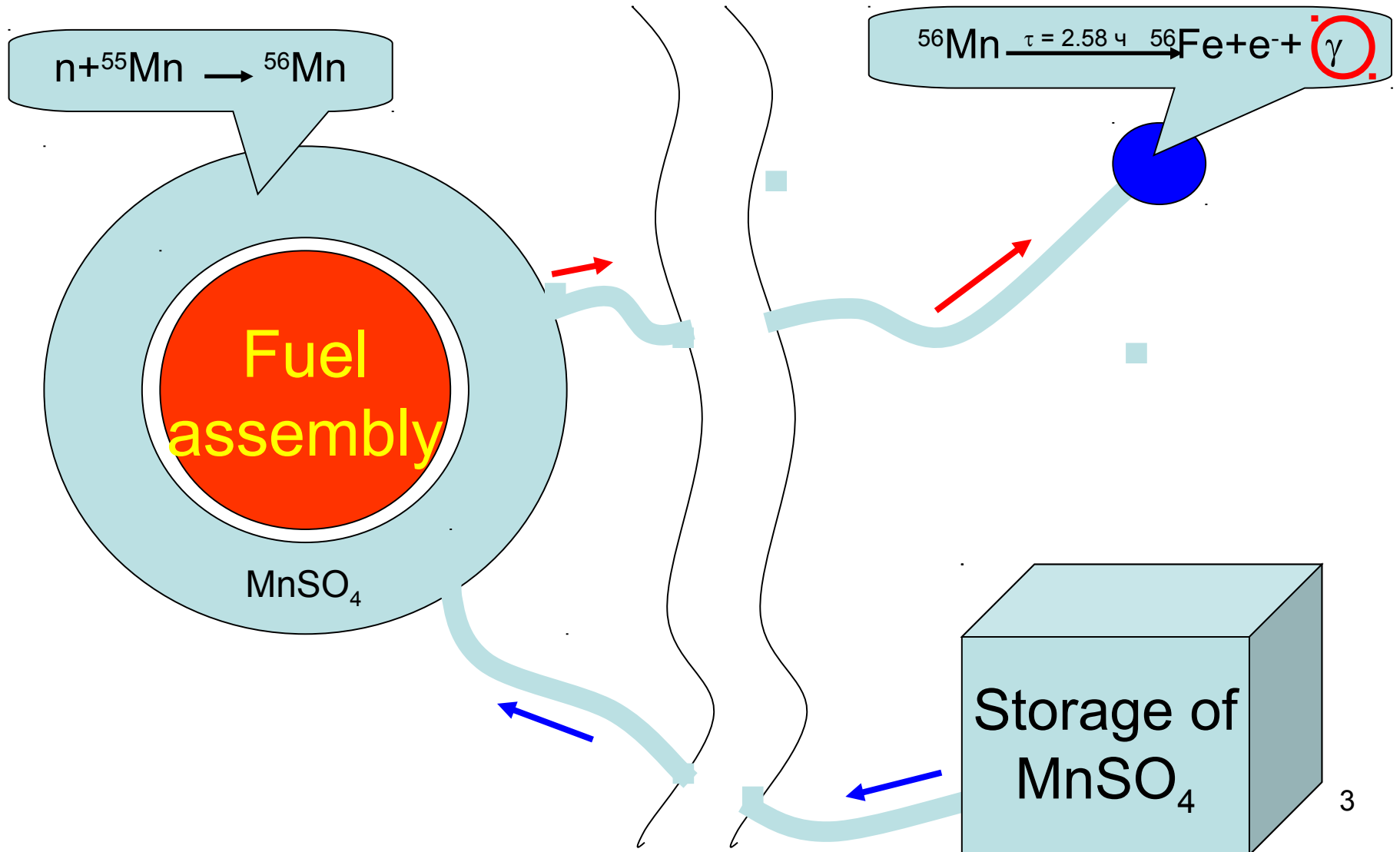
# Determination of spent fuel characteristics

Method	Plus	Minus
radiochemistry	accuracy ~ 1%	labor intensive
$\gamma$ -spectroscopy	fast	influence of $\gamma$ -background
fork method	fast	influence of $\gamma$ -background

# Neutron activation analysis

error ~ 3-4%

$E_\gamma = 0.847, 1.811, 2.11 \text{ MeV}$



# Key questions

Does the axial distribution influence measurements?

Can the concentration of actinides be related to neutron intensity?

# Calculation of an infinite lattice of fuel assemblies

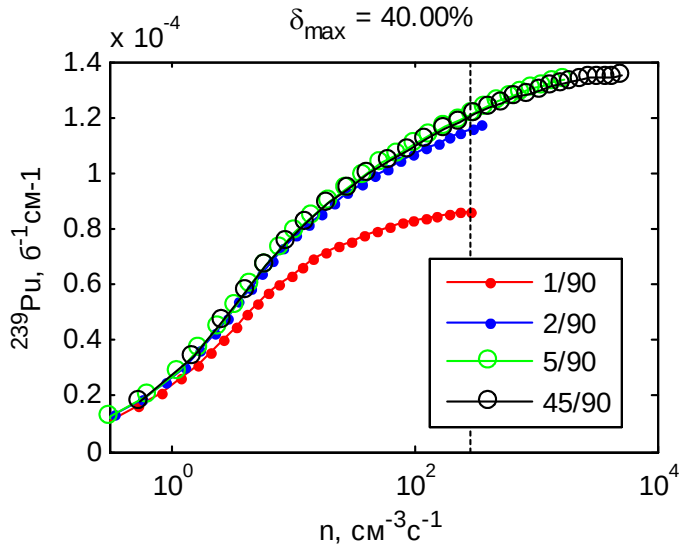
- Enrichment 4.4%
- Time: 890 effective days (~2.5 years)
- 90 layers over height
- Absorber withdrawal: 0 and 100%
- $\text{H}_3\text{BO}_3$  content: 0 and 5 g/kg

## Neutron yield from actinide decay

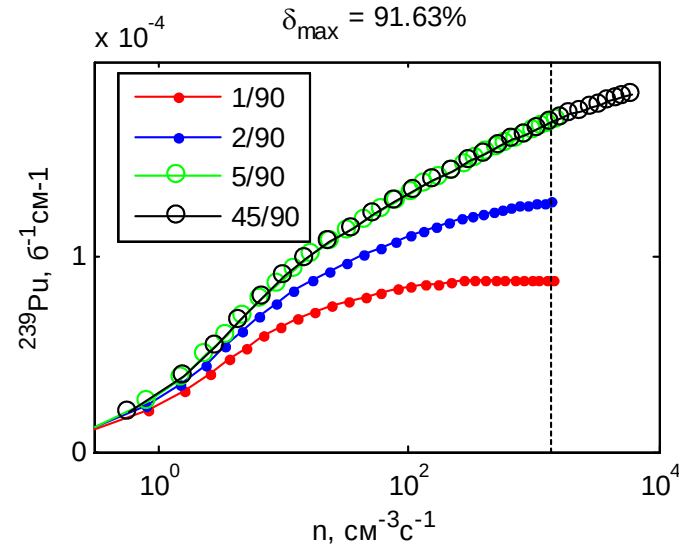
	$^{238}\text{Pu}$	$^{239}\text{Pu}$	$^{240}\text{Pu}$	$^{242}\text{Pu}$	$^{244}\text{Cm}$
fission, $\text{n s}^{-1}\text{g}^{-1}$	$2,61 \cdot 10^3$	0,022	$1,02 \cdot 10^3$	$1,72 \cdot 10^3$	$1,08 \cdot 10^7$
$\alpha$ -decay, $\text{n s}^{-1}\text{g}^{-1}$	$1,33 \cdot 10^4$	38,2	141	2,09	$7,71 \cdot 10^4$

# $^{239}\text{Pu}$ concentration versus neutron intensity at different heights

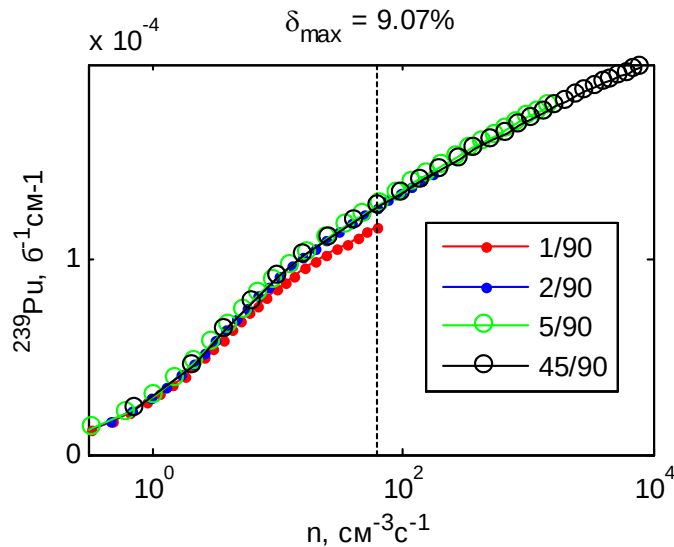
$h = 100\%$   
 $cB = 0$   
 $g/kg$   
 $\delta = 40\%$



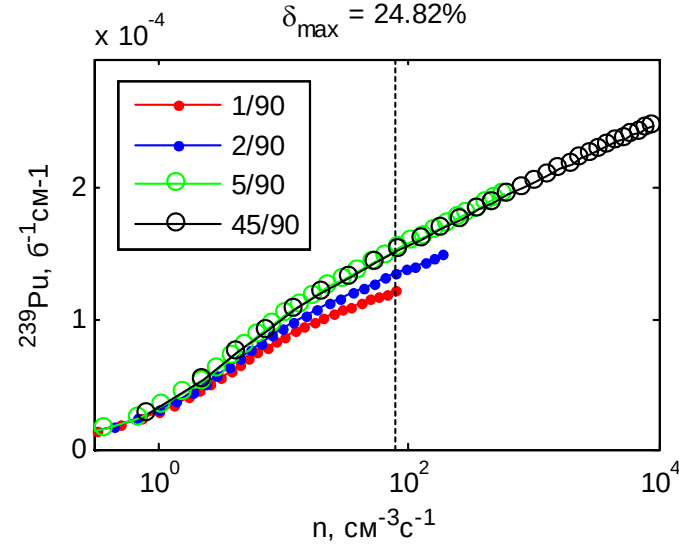
$h = 0\%$   
 $cB = 0$   
 $g/kg$   
 $\delta = 92\%$



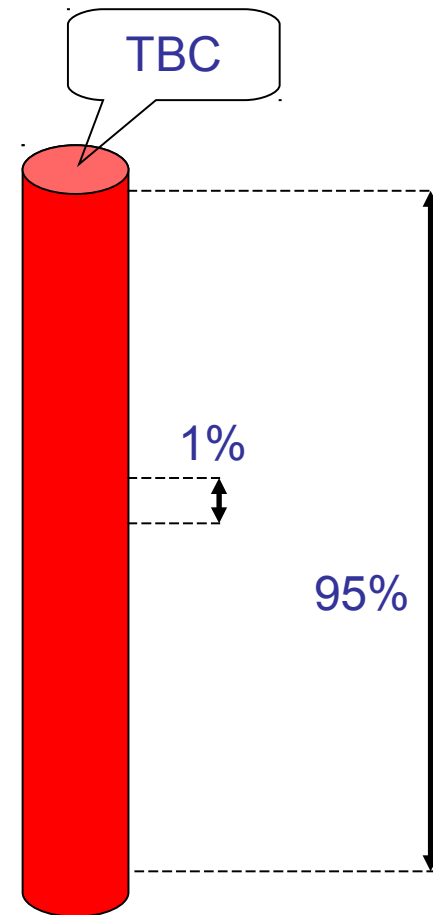
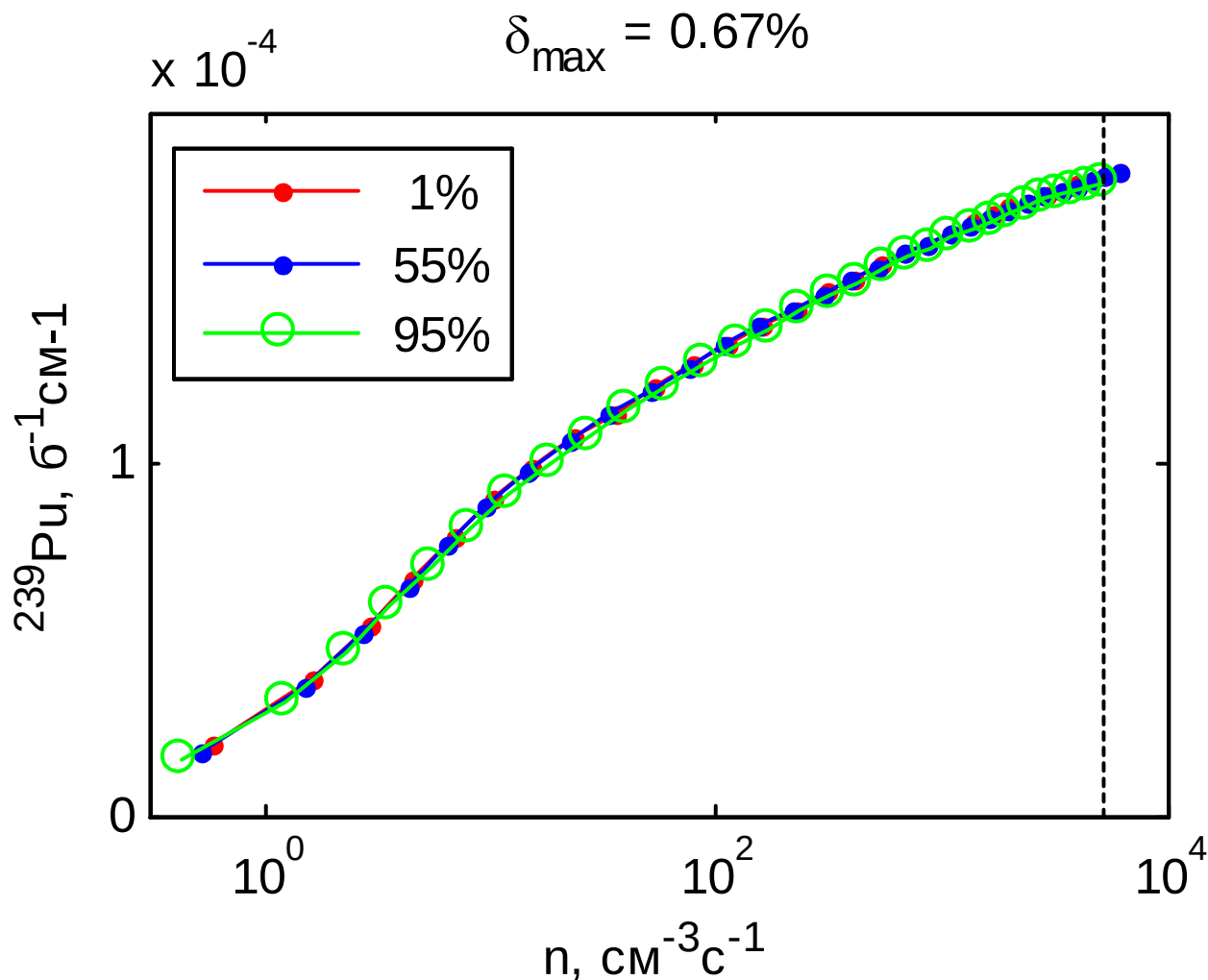
$h = 100\%$   
 $cB = 5$   
 $g/kg$   
 $\delta = 9\%$



$h = 0\%$   
 $cB = 5$   
 $g/kg$   
 $\delta = 23\%$

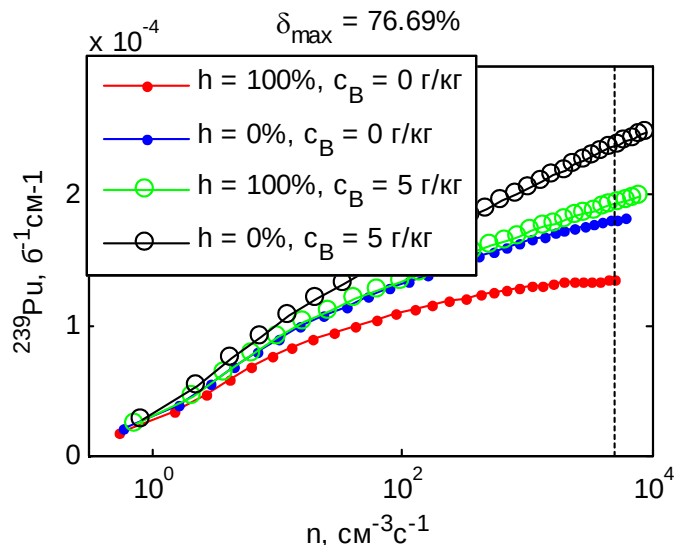


# $^{239}\text{Pu}$ concentration versus intensity measured over different height sections ( $h = 0\%$ , $c_B = 0 \text{ g/kg}$ )

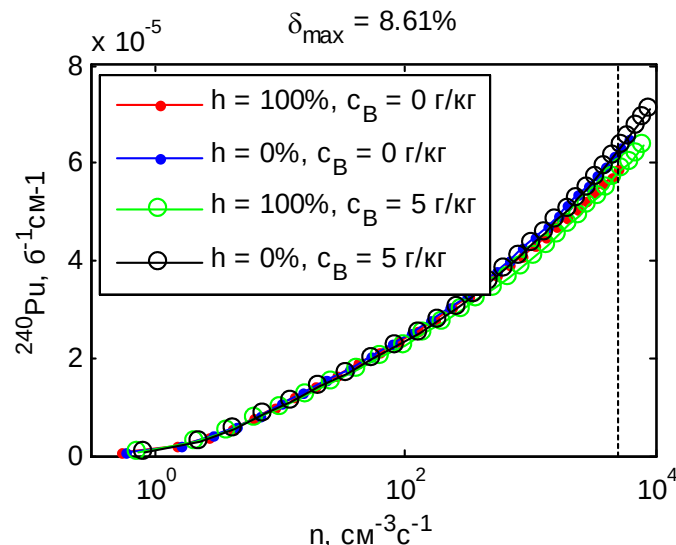


# Plutonium concentrations versus neutron intensity

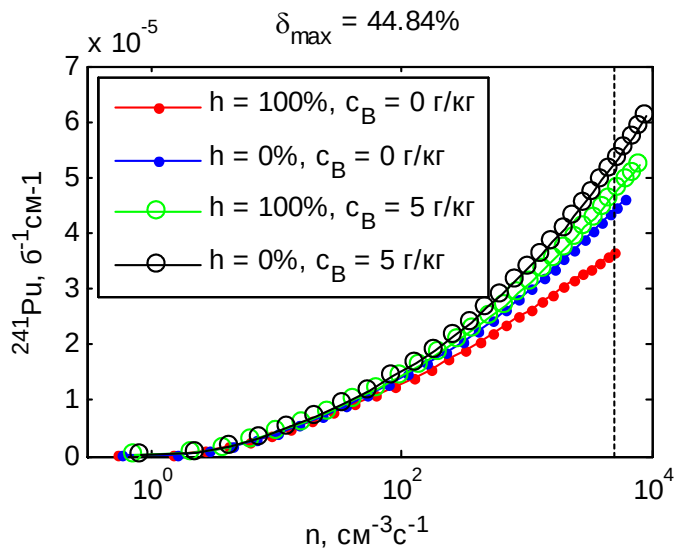
$^{239}\text{Pu}$   
 $\delta = 77\%$



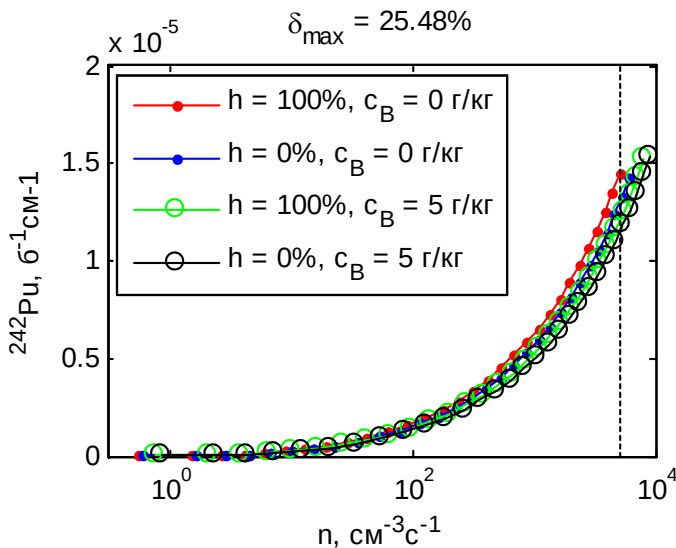
$^{240}\text{Pu}$   
 $\delta = 9\%$



$^{241}\text{Pu}$   
 $\delta = 45\%$



$^{242}\text{Pu}$   
 $\delta = 25\%$



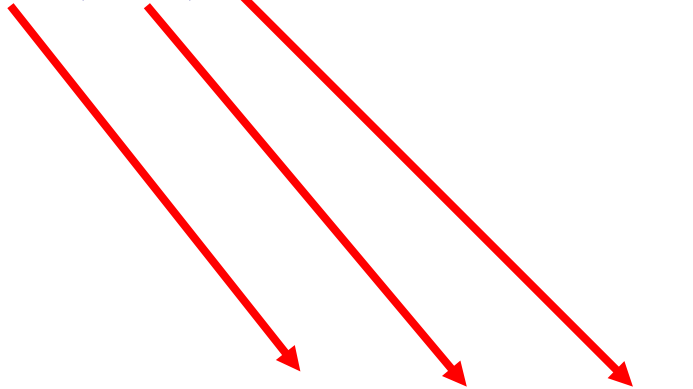


# VVER-1000 campaign calculation

- Enrichment 1.6, 2.4, 3.6%
- Time: 300 effective days
- Thermal hydraulics
- Boron control of criticality
- No burnable absorber

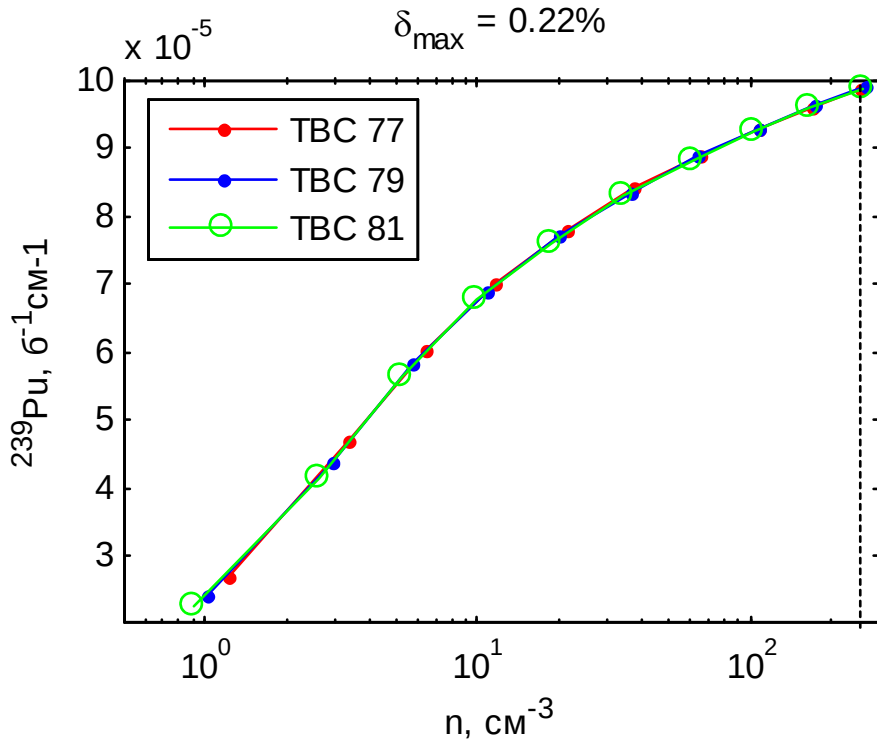
# Fuel column cartogram Block 1 of Volgodonsk NPP

FA 77, 79, 81

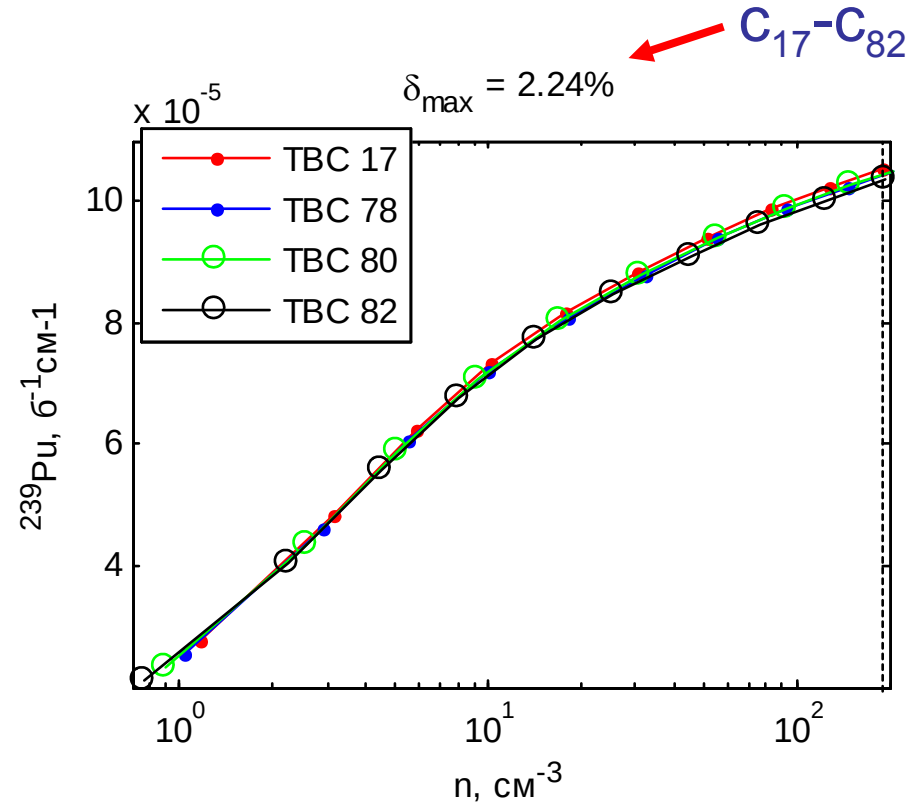


FA 78, 80, 82, 17

# Effect of position in core

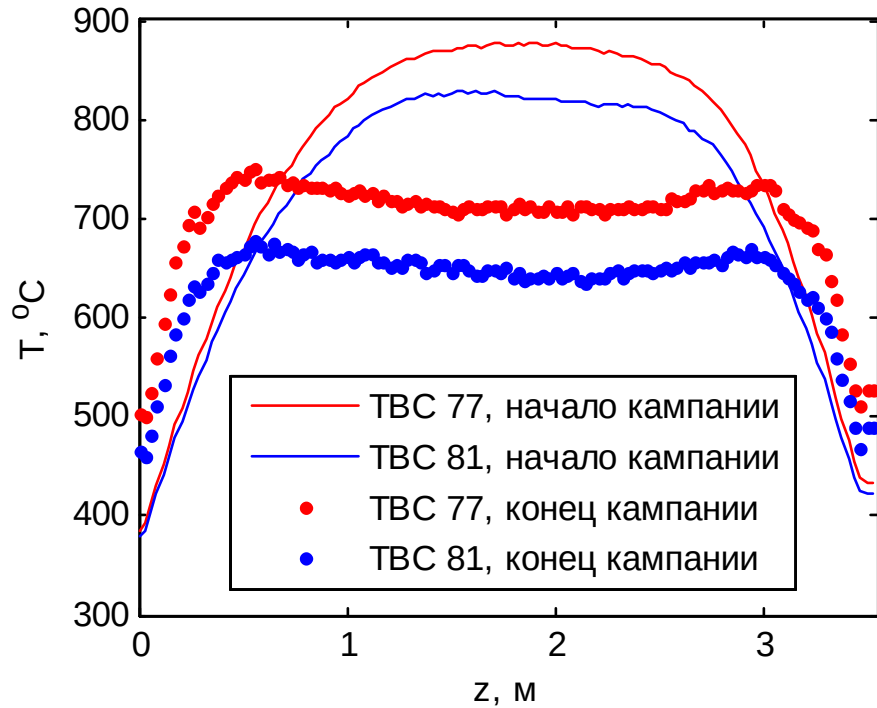


$^{239}\text{Pu}$  concentration  
Enrichment 1.6%

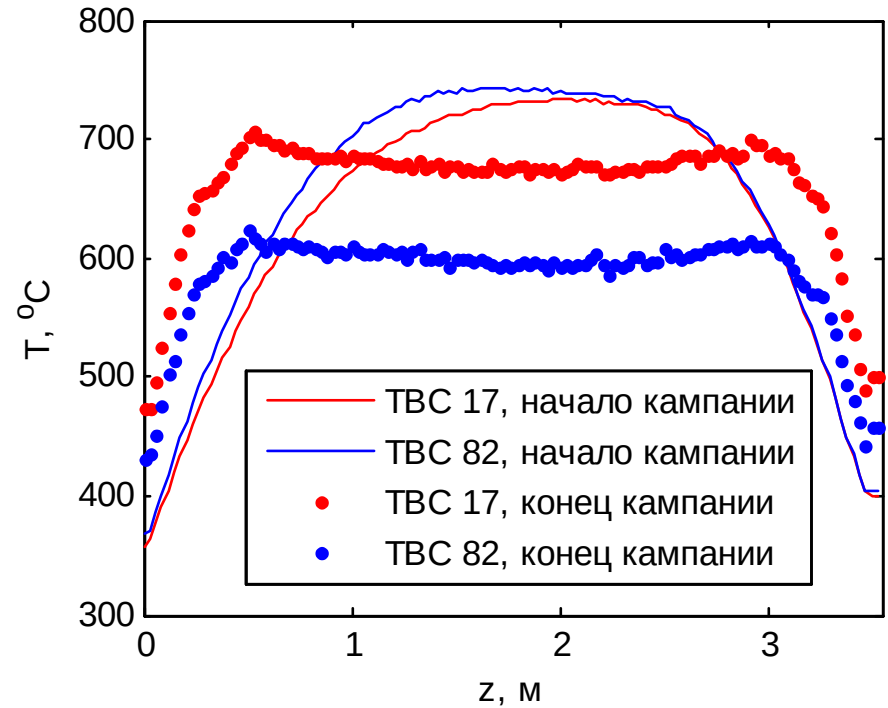


$^{239}\text{Pu}$  concentration  
Enrichment 2.4%

# Average fuel temperature

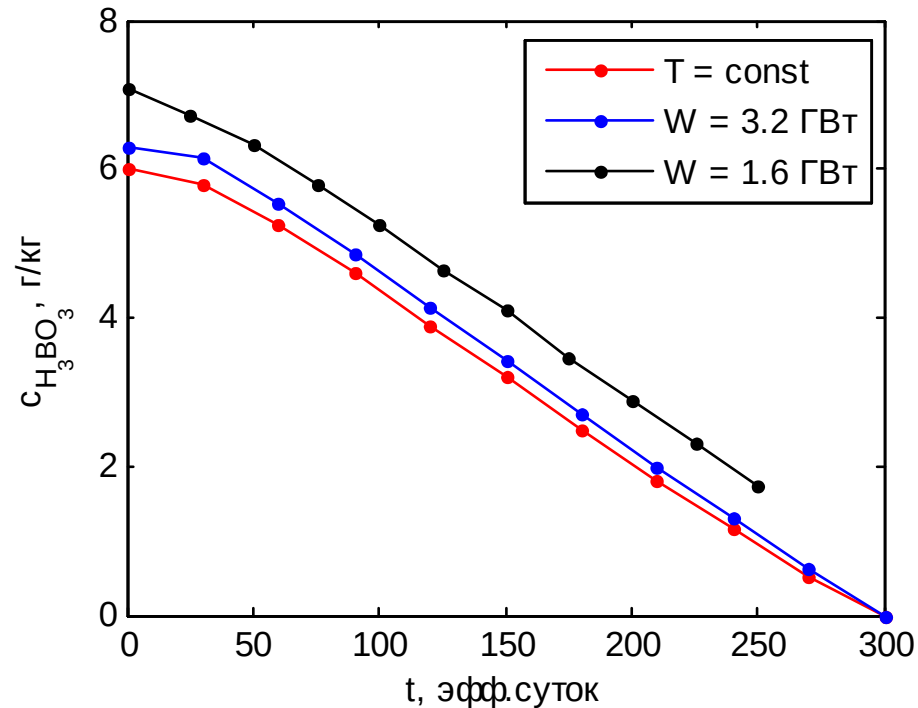


Average fuel temperature  
Enrichment 1.6%

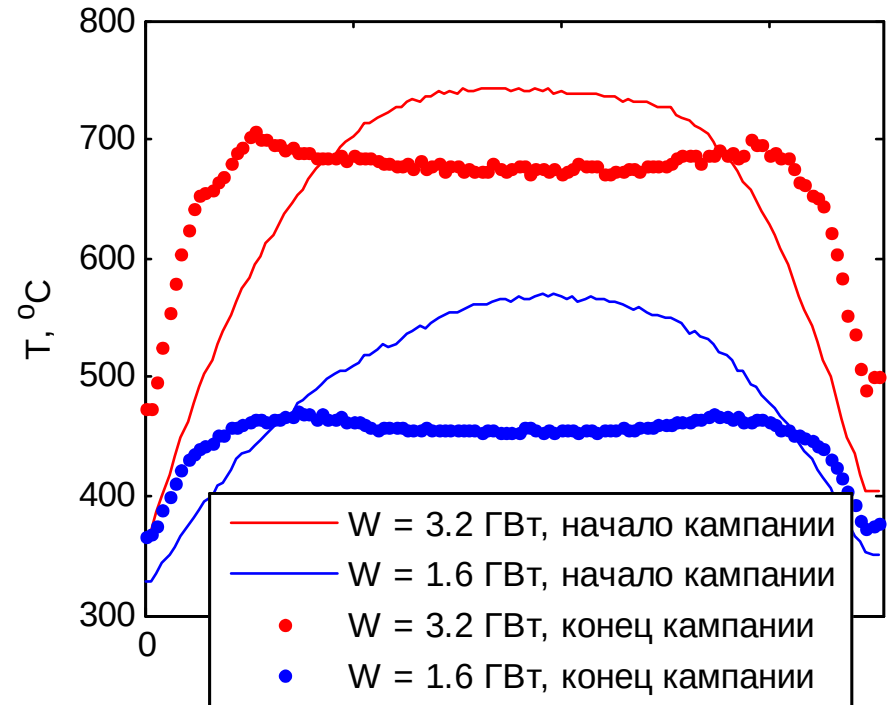


Average fuel temperature  
Enrichment 2.4%

# The effect of operating conditions

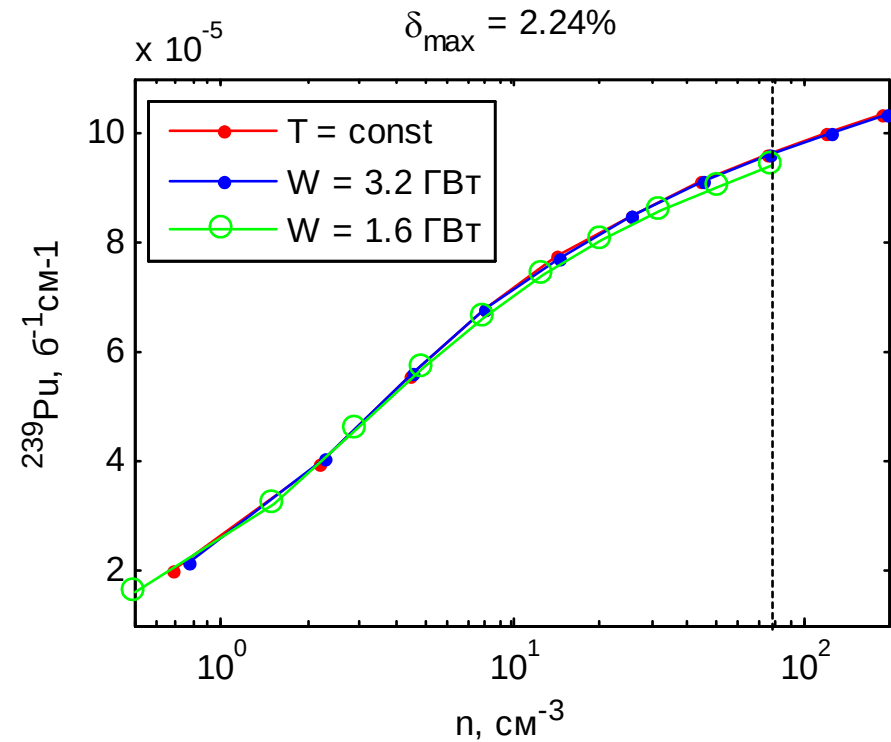


Boron concentration

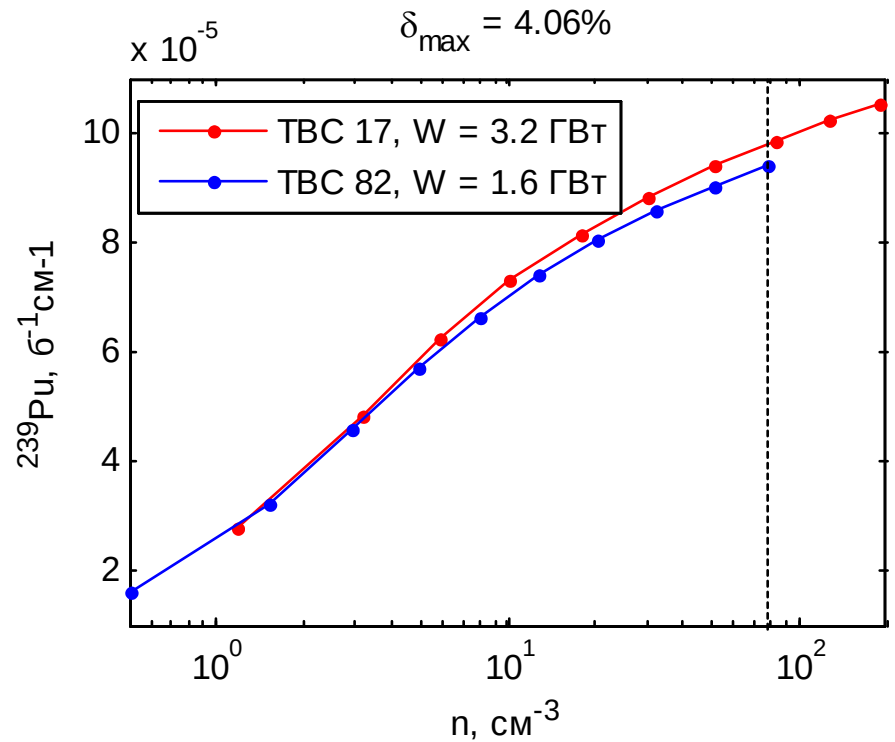


Average fuel temperature  
in FA 82

# The effect of operating conditions



$^{239}\text{Pu}$  concentration  
in FA 82

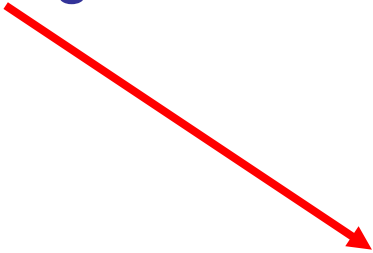


$^{239}\text{Pu}$  concentration  
in FA 17 and 82

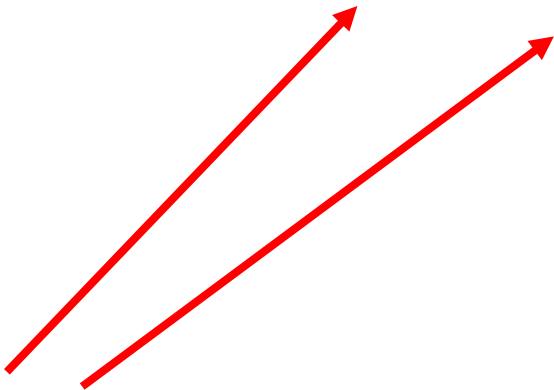
# The effect of loading

FA with enrichment 1.6% are replaced by FA with enrichment 3.6%

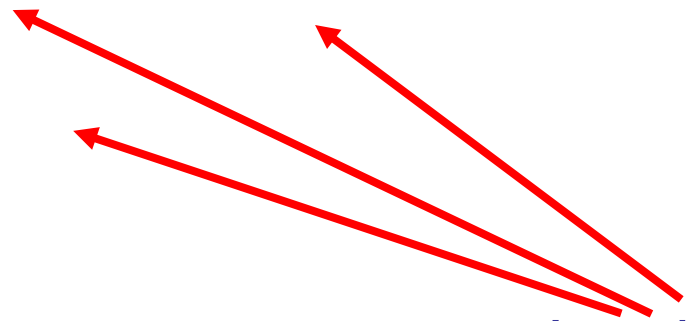
Loading 1



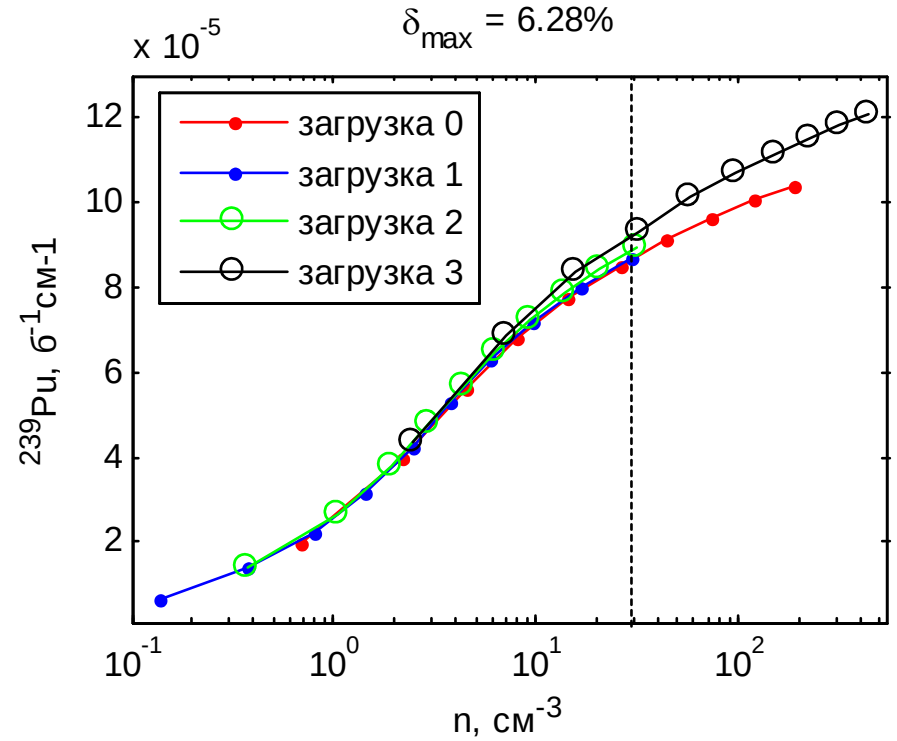
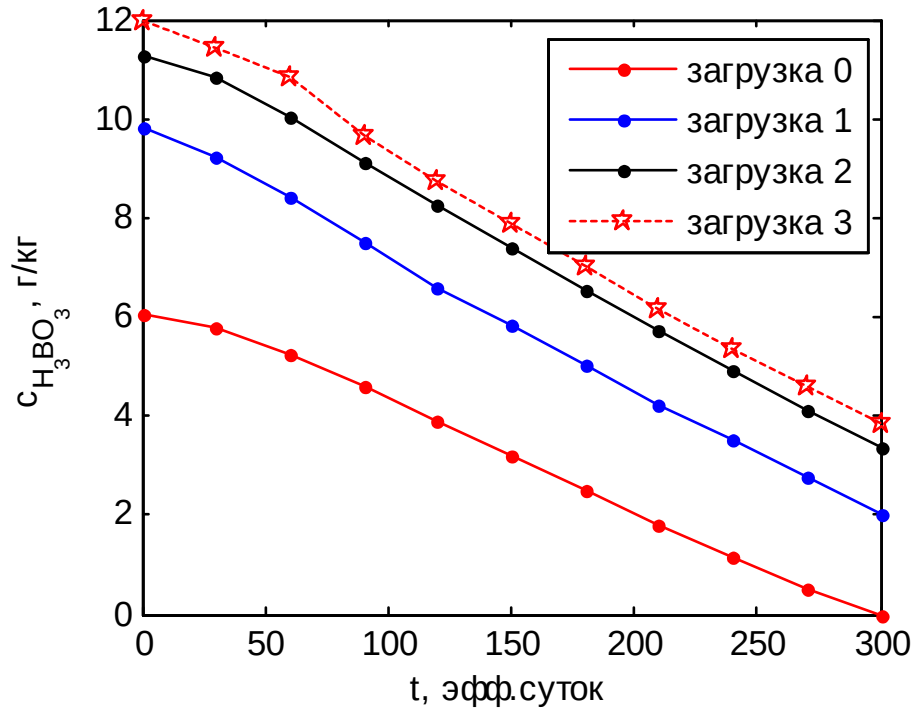
Loading 2



Loading 3



# The effect of loading

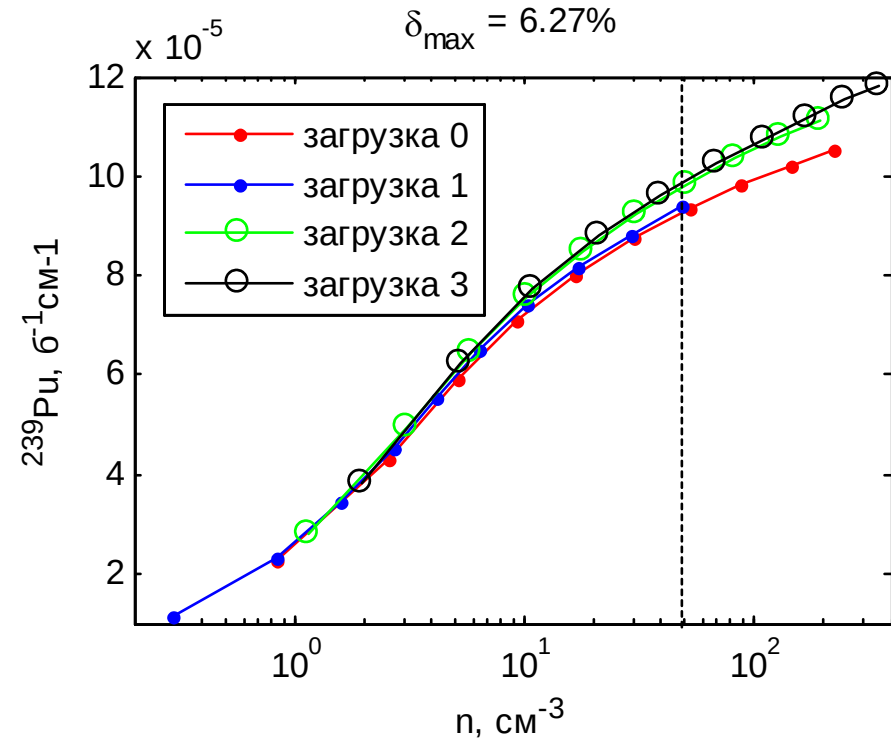


Boron concentration

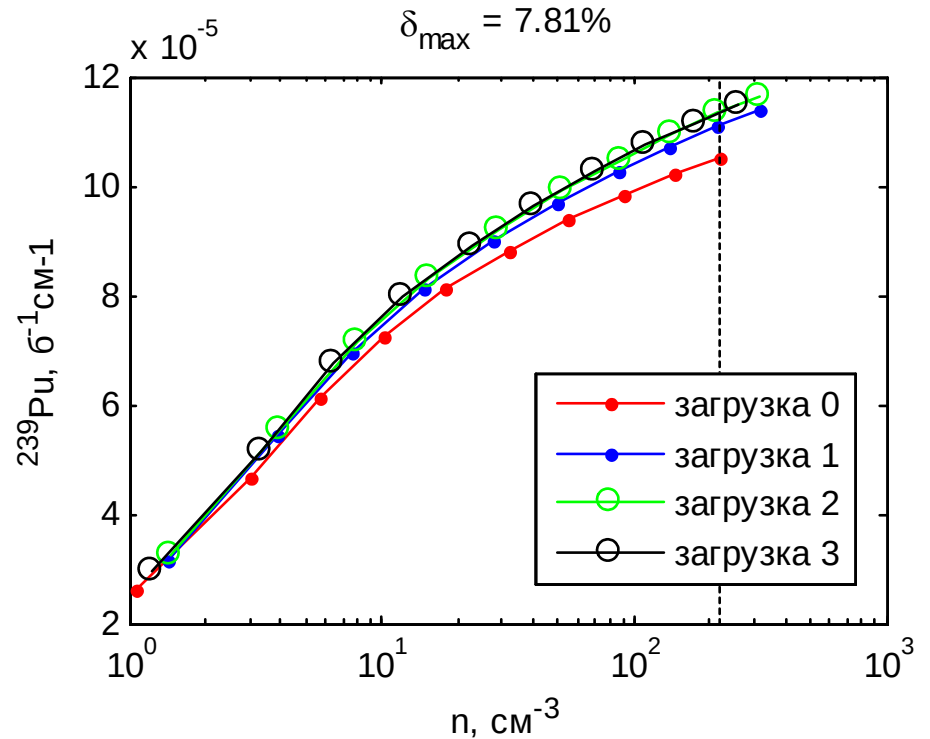
$^{239}\text{Pu}$  concentration  
in FA 82



# The effect of loading



$^{239}\text{Pu}$  concentration  
in FA 80



$^{239}\text{Pu}$  concentration  
in FA 78

# Conclusion

- Position in core: ~ 2%
- Operating conditions (upper estimate): ~ 2%; quite realistic → 0
- Core loading: highest uncertainty up to 8%  
it possible to level it?
- With the measurement error of about 3-4%, the overall error is optimistically estimated to be ~ 5-6%.